#### Multiphysics Modeling Using the MOOSE Framework

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IAEA Workshop on Multiphysics Modelling to Optimize Design and Safety of Advanced Nuclear Reactors

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#### Outline

## I

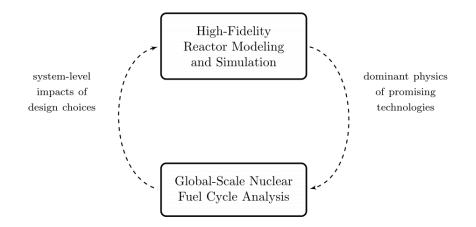
#### 1 Different Problems, Different Solutions

**2** MOOSE Framework

**3** Moltres (a MOOSE Application)

#### Insights at Disparate Scales





#### Challenges in Liquid-Fueled Reactor Simulation

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- ① Contemporary burnup codes cannot treat fuel movement.
- 2 Neutron precursor locations drift before neutron emission.
- **③** Operational and safety parameters change during reactor operation.
- (a) Neutronics and thermal hydraulics are tightly interdependent.

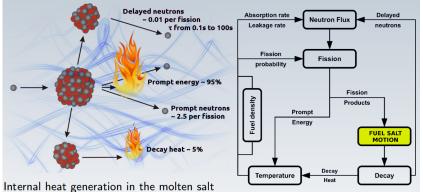


Figure: Challenges in simulating MSRs (Image courtesy of Manuele Aufiero, 2012).

#### Outline

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#### Different Problems, Different Solutions

#### **2** MOOSE Framework

**3** Moltres (a MOOSE Application)

#### MOOSE Framework

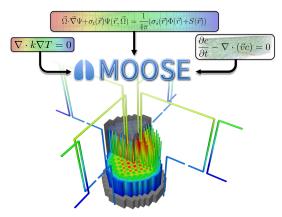


Figure: Multi-physics Object-Oriented Simulation Environment (MOOSE) [1, 5, 4].

- Developed by Idaho National Laboratory [1, 5, 4]
- Framework is truly open source (LGPL)
- Accessible docs and tutorials at https: //mooseframework. inl.gov
- Source code at https://github.com/ idaholab/moose

#### MOOSE Apps & Kernels

## $\vec{\Omega} \cdot \vec{\nabla} \Psi + \sigma_t(\vec{r}) \Psi(\vec{r}, \vec{\Omega}) = \frac{1}{4\pi} (\sigma_s(\vec{r}) \Phi(\vec{r}) + S(\vec{r}))$ $\partial c$ $\nabla \cdot k \nabla T = 0$ $\nabla \cdot (\vec{v}c) = 0$ $\overline{\partial t}$

Figure: Multi-physics Object-Oriented Simulation Environment (MOOSE).

#### Lots of Open Kernels

- Chemical reactions
- Contact
- Fluid Properties
- Functional Expansion Tools
- Geochemistry
- Heat Conduction
- Level Set
- Navier-Stokes
- Peridynamics
- Phase Field
- Porous Flow
- Ray Tracing
- Reconstructed Discontinous Galerkin
- Tensor Mechanics
- . . .

#### MOOSE Apps & Kernels

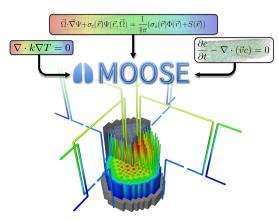
# $\vec{\Omega} \cdot \vec{\nabla} \Psi + \sigma_t(\vec{r}) \Psi(\vec{r}, \vec{\Omega}) = \frac{1}{4\pi} (\sigma_s(\vec{r}) \Phi(\vec{r}) + S(\vec{r}))$ $\frac{\partial c}{\partial t}$ $\nabla \cdot k \nabla T = 0$ $\nabla \cdot (\vec{v}c) = 0$

Figure: Multi-physics Object-Oriented Simulation Environment (MOOSE).

#### Lots of Open Apps

- Moltres (MSRs) [?]
- Squirrel (Utilities)
- Mastodon (structural dynamics, seismology)
- pika (microstructure)
- falcon
- blackbear
- crane (plasma chemistry)
- WhALE (fluid-structure mechanics)

#### MOOSE Apps & Kernels



#### Lots of Restricted Apps

- BISON
- Marmot
- RattleSNake
- Pronghorn
- RELAP-7

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Figure: Multi-physics Object-Oriented Simulation Environment (MOOSE).

#### How Does it Work?



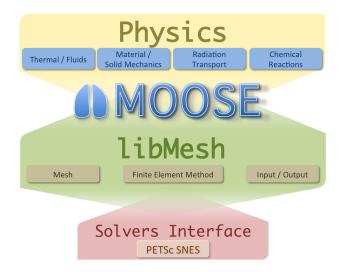


Figure: Shamelessly copied from the MOOSE Team Workshop slides.

#### How Does it Work?

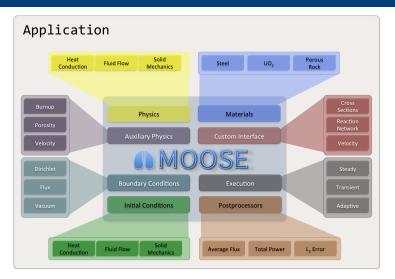
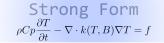
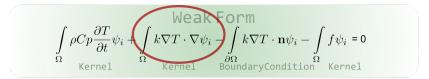


Figure: Shamelessly copied from the MOOSE Team Workshop slides.

#### How Does it Work?







### Actual Code

return \_k[\_qp]\*\_grad\_u[\_qp]\*\_grad\_test[\_i][\_qp];

Figure: Shamelessly copied from the MOOSE Team Workshop slides.

#### MOOSE: Key Features

- MOOSE Framework is truly open source (LGPL)
- Developed initially for nuclear applications
- Signficant long-term support from US DOE
- Continuous integration support (CIVET)
- Intuitive parallel multiscale solves
- Easy developer onboarding

- Object Oriented, C++
- Interfaces with libMesh to discretize simulation volume into finite elements
- Residuals and Jacobians handed off to PetSc which handles solution of resulting non-linear system of algebraic equations
- Fully-coupled, fully-implicit multiphysics solver
- Automatically parallel (largest runs >100,000 CPU cores!)
- Built-in adaptive meshing & timestepping

#### Pros and Cons

#### Pros (+)

- LGPL means the Framework is open, but apps can be restricted
- Vast array of available apps and kernels
- Many solver and preconditioning options
- Finite Element Modeling
- Full coupling is optional
- Generates gorgeous visualizations

#### Cons (-)

- LGPL means the Framework is open, but apps can be restricted
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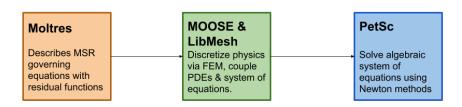
#### Different Problems, Different Solutions

**2** MOOSE Framework

**3** Moltres (a MOOSE Application)

#### Moltres: Coupling in MOOSE





#### Moltres: Basics

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- Developed in ARFC group
- Fluid-fuelled, molten salt reactors
- Multi-group diffusion (arbitrary groups)
- Advective movement of delayed neutron precursors
- Navier-Stokes thermal hydraulics
- 3D unstructured
- 2D axisymmetric
- 3D structured
- Initial developer: Alexander Lindsay [3]

#### Acquiring Moltres



```
git clone https://github.com/arfc/moltres
cd moltres
git submodule init
git submodule update
```

#### Diffusion in Moltres

$$\frac{1}{v_g}\frac{\partial\phi_g}{\partial t} - \nabla \cdot D_g \nabla\phi_g + \Sigma_g' \phi_g =$$
(1)

$$\sum_{g\neq g'}^{G} \Sigma_{g'\rightarrow g}^{s} \phi_{g'} + \chi_{g}^{\rho} \sum_{g'=1}^{G} (1-\beta) \nu \Sigma_{g'}^{f} \phi_{g'} + \chi_{g}^{d} \sum_{i}^{l} \lambda_{i} C_{i}$$

$$\tag{2}$$

- $v_g$  = speed of neutrons in group g
- $\phi_{g} =$ flux of neutrons in group g
  - t = time
- $D_g$  = Diffusion coefficient for neutrons in group g
- $\Sigma_{\sigma}^{r}$  = macroscopic cross-section for

removal of neutrons from group g

$$\sum_{g' \to g}^{s} =$$
 macroscopic cross-section of  
scattering from g' to g

 $\chi^{p}_{g}$  = prompt fission spectrum, neutrons in group g

- G = number of discrete groups, g
- $\nu =$  neutrons produced per fission
- $\Sigma_g^f$  = macroscopic fission cross section due to neutrons in group g
- $\chi_g^d = delayed$  neutrons in group g
  - I = delayed neutron precursor groups
- $\beta = delayed$  neutron fraction
- $\lambda_i$  = average decay constant

of delayed neutron precursors in group i

 $C_i$  = concentration of delayed neutron

precursors in precursor group i

#### Moltres Delayed Neutrons

$$\frac{\partial C_i}{\partial t} = \sum_{g'=1}^{6} \beta_i \nu \Sigma_{g'}^{f} \phi_{g'} - \lambda_i C_i - \frac{\partial}{\partial z} u C_i$$
(3)

 $\begin{aligned} G &= \text{number of discrete groups, g} \\ I &= \text{ delayed neutron precursor groups} \\ C_i &= \text{ concentration of delayed neutron} \\ \text{ precursors in precursor group i} \\ .u &= \text{ vertical fluid velocity} \\ \lambda_i &= \text{ average decay constant} \\ \text{ of delayed neutron precursors in group i} \\ \beta &= \text{ fraction of delayed neutron} \end{aligned}$ 

precursors in group i

#### Moltres Fuel Temperature



$$\rho_f c_{\rho,f} \frac{\partial T_f}{\partial t} + \nabla \cdot \left( \rho_f c_{\rho,f} \vec{u} \cdot T_f - k_f \nabla T_f \right) = Q_f \tag{4}$$

$$\rho_f = \text{density of fuel salt}$$
(5)

$$c_{p,f}$$
 = specific heat capacity of fuel salt (6)

$$T_f$$
 = temperature of fuel salt (7)

$$\vec{u} =$$
velocity of fuel salt (8)

$$k_f =$$
thermal conductivity of fuel salt (9)

$$Q_f = \text{source term} = \sum_{g=1}^{G} \epsilon_{f,g} \Sigma_{f,g} \phi_g$$
(10)

#### Moltres Moderator Temperature



## $\rho_g c_{p,g} \frac{\partial T_g}{\partial t} + \nabla \cdot (-k_g \nabla T_g) = Q_g \tag{11}$

(12)

$ ho_{ m g}={ m density}$ of graphite moderator	(13)
$c_{p,g} =$ specific heat capacity of graphite moderator	(14)
$T_g =$ temperature of graphite moderator	(15)
$k_g =$ thermal conductivity of graphite moderator	(16)
$Q_g$ = source term in graphite moderator	(17)
	(18)

#### Moltres MSRE Simulation

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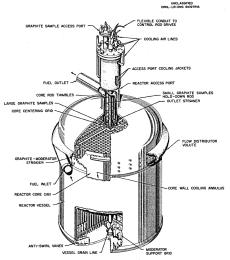
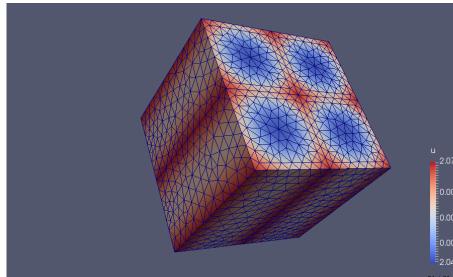


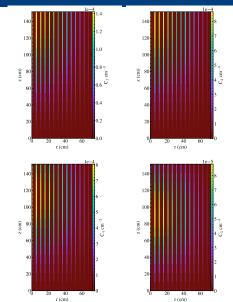
Fig. 6. MSRE Reactor Vessel.

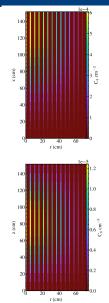
#### Mesh Generation for MOOSE Apps like Moltres



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#### Moltres Precursor Drift





#### Moltres: More Complex Mesh

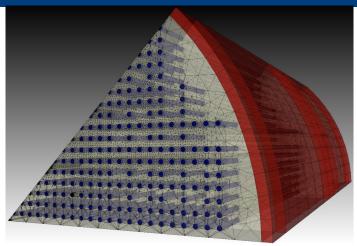


Figure: TAP Mesh generated by Alvin Lee [2]. Red = Reactor Vessel Wall, Light Yellow = Fuel Salt, Dark Gray = Control Rods, Blue = Fuel Salt radially co-located with the Moderator Rods.

#### Moltres: Multiphysics simulation (3D)

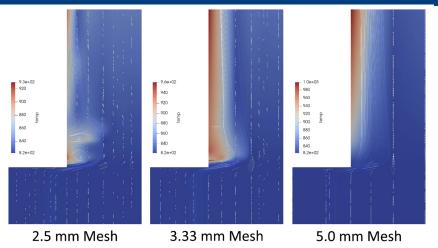


Figure: Meshing study by Alvin Lee  $\left[2\right]$  regarding KH instabilities and resolution of MSR fuel salt vortices.

#### What now?

## I

#### Get Started

https://mooseframework.inl.gov/ https://github.com/idaholab/moose https://github.com/arfc/moltres **Other Tools** https://gmsh.info

https://github.com/pyne/pyne https://github.com/openmc-dev/openmc https://www.paraview.org

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- The authors would like to thank members of Advanced Reactors and Fuel Cycles research group (ARFC) at the University of Illinois at Urbana Champaign who provided valuable code reviews and proofreading.
- This work is derived from the work of Alex Lindsay (Idaho National Laboratory), Gavin Ridley (University of Tennessee-Knoxville), Andrei Rykhlevskii (Argonne National Laboratory), Sun Myung Park (UIUC), and Alvin Lee (UIUC).



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